

Lamarsh solution
chap7



LAMARSH SOLUTIONS CHAPTER-7 PART-1

7. 1 Look at example 7. 1 in the textbook, only the moderator materials are different Since the reactor is critical, $k_{eff} = 1$? $T_{1/2} = 2.065$ from table 6. 3 so $f = 0.484$ We will use $t_d = t_{dM} (1/f)$ and t_{dM} from table 7. 1 t_{dM} , $D_{2O} = 4.3e-2$; t_{dM} , $B_e = 3.9e-3$; t_{dM} , $C = 0.017$ Then, $t_d = D_{2O} = 0.022188\text{sec}$; t_{dM} , $B_e = 2.0124e-3\text{sec}$; t_{dM} , $C = 8.772e-3\text{sec}$

7. 5 One? delayed? neutron group reactivity equation; ?? λ_p ?? λ_p ?? where ?? 0.0065 ; ?? 0.1sec ? ?? 1.1 ?? λ_p ?? For $\lambda_p = 0.0\text{sec}$ For $\lambda_p = 0.0001\text{sec}$ For $\lambda_p = 0.001\text{sec}$ Note: In this question examine the figure 7. and see that to give a constant period value , say 1 sec, you should give much more reactivity as p. neutron lifetime increases.

And it is strongly recommended that before exam, study figure 7. 1 . 7. 8 ?? $2e-4$ from figure 7. 2 so you can ignore jump in power(flux) in this positive reactivity insertion situation $t = P_f / P_i e T$ then $t = \ln f / T = 3.456\text{hr}$ $P_i = 7.10$

$\ln eq 7. 19$ prompt neutrons: $(1-\lambda_p)k_{eff}^2 a / T$ delayed neutrons: $p C$ in a critical reactor(from 7. 21) $k_{eff}^2 dC / (0.0065 a T) p C$ $k_{eff}^2 a / T dt p$ $s T (1-\lambda_p)k_{eff}^2 a / T k_{eff}^2 a / T ????????$ prompt delayed

Now you can compare their values prompt $(1-\lambda_p)k_{eff}^2 a / T$ delayed $p C$ LAMARSH SOLUTIONS CHAPTER-7 PART-2

7. 12 ?? P_0 ? ??? t ? ?? 1 $P(t)$? e in here ?? then, and ?? ??? ??? T t P_0 T $P(t)$? e in here take $T=-80\text{sec}$?? 1 ? ?? t ? P_0 P_0 ? 10 ? e 80 ? t ? 25.24 min . 1 ? (5) ? $9.7.14$ k ? , 0 ? ?? p_f 0 , critical state k ? , 1 ? ?? p_f 1 , original state ?? k ? , 1 ? 1 k ? , 1 ? k ? , 1 ? k ? , 0 k ? , 1 ? ?? p_f 0 ? ?? p_f 0 f ? 1 ? 0 ? ?? p_f 1 ? a_{1F} ? $a_0 F$ f_1 ? $F f_0$? and we know $a_{1F} = 0.95$? $a_0 F$ and finally, $M F M$? a_{1F} ? ?? $a_0 F$? ?? a_f ? 1 ? 0.95 ? $a_0 F$? ?? a_M ? ?? 1 ? (f_1) 0.95 ? $a_0 F$? ?? a_M 7.16 20 min ? $60\text{sec}/\text{min}$? 1731.6sec . $\ln 2$)From fig 7. 2 reactivity is small so small reactivity assumption can be used

as, $\frac{1}{1} T = ? ? i t i ?? ? ? 0.0848$ (from table 7. 3)= $4.89e-5 = 4.89e-3\% ? i$
 1731.6 $4.89e-5$ also in dollars= $? 7.52e ? 3\$? 0.752$ cents $0.0065(U235) t$
 $T a) P_0 ? P_0 e ? T ? 7.17$ 8hr ? 60 min? 60sec 8hr ? 60 min? 60sec ? $T ? ? 6253.8$ sec(very large) $T \ln 100$ b) We will make small reactivity insertion approximation using the insight given by figure 7. 2 for U-235 so, $\frac{1}{1} T = ? ? i t i ?? ? ? 0.0324$ (from table 7. 3)= $5.18e-6 ? i 6253.8$ a) $100\text{MW} ? 1\text{MWe}$ 7. 18 a) From fig 7. 1 when $? ? 0 ? 1 ? 0$ so $T = 1 ? T ?? ? 1$ b) Use prompt jump approximation, $t t$

$P_0 ? T P_0 T 10\text{watts}$ $(300 ? 100)\text{sec}$ $P(t) = e ? e ? e 100\text{sec} ? 82\text{watts} ? 0.099$??? 1? 1? ? 1 c) Use $T = -80\text{sec}$. 300sec $t t P_0 ? T P_0 T 82\text{watts} ? (t ? 80\text{sec})$ $P(t) = e ? e ? e ? 8 ??? 1? 1 ? (?) ? 1$ LAMARSH SOLUTIONS CHAPTER-7 PART-3
7. 20 Insert 7. 56 into 7. 57 and plot reactivity vs rod radius Using eq. 7. 57 and 7. 56 we plotted and found the radius value for 10% reactivity= 3. 9 cm reactivity vs rod radius(a) 0. 14 0. 12 X: 3. 9 Y: 0. 1004 reactivity 0. 1 0. 08 0. 06 0. 04 0. 02 0 0 0. 5 1 1. 5 2 2. 5 rod radius 3 3. 5 4 4. 5 5 7. 23 a) For a slab this equation is solved you know as, $x x q ? T(x) ? A_1 \sinh(?) ? A_2 \cosh(?)$?

T then to find the constants you must introduce $L L ? a$ 2 boundary conditions 1 $d ? T 1 d ? T 1$ B. C. 1: ? 0 @ $x = 0$ and B. C. 2: ? ? @ $x = (m/2) - a$? $T dx ? T dx d$ Introducing B. C. 1 you find $A_1 ? 0$ and B. C. 2 $x ? ? \cosh(?) ? ? q L A_2 = -T ? 1 ? ? d ? a ? \sinh((m ? 2a) / 2L) ? \cosh((m ? 2a) / 2L) ? ? L ?$ So finally, $x ? ? \cosh(?) ? ? q T L ? T(x) ? ? 1 ? ? d ? a ? \sinh((m ? 2a) / 2L) ? \cosh((m ? 2a) / 2L) ? ? L ?$ b) Neutron current density at the blade surface, $d ? L J @ (m/2) - a ? ? D T ? d dx @ (m/2) - a ? \coth((m ? 2a) / 2L) L$ Let's follow the instructions in the question Multiply the n. current density by the area of the

blades in the cell... --What is the area of the blades in the cell: From fig 7. 9, assume unit depth into the page so the cross sectional area of one of four blades, $A=(l-a) \cdot 1$ Divide by the total number of neutrons thermalizing per second in the cell ---What is the volume of the cell: From fig 7. 9, assume unit depth into the page so $V=(m \cdot 2a) \cdot (m \cdot 2a) \cdot 1$ So as in page 358 4(l ? a) 1 fR ? 2 (m ? 2a) d ? coth((m ? 2a) / 2L) L 7. 25 You should find the B-10 average atom density in the reactor Total mass of B-10= 50rods ? 500g= 25 ? 103g 25e3 N? ? 0. 6022e24 ? 1. 39e27atoms 10. 8 Atom density averaged over whole reactor volume, 1. 39e27 NB ? ? 2. e21 atoms/cm³ ? ? aB ? 2. 9e21? 0. 27b ? 7. 8e ? 4cm ? 1 4 ?(48. 5)3 3 7. 8e ? 4 ? use eq. 7. 62 then find,? w ? ? 0. 0938 ? 9. 4% 0. 00833 ? 0. 000019 7. 27 H ? 100cm and ?? ? ? 0.x ? H a) For x ? 3H / 4 ? 75cm 1 ? x ? ? Sin(2? x / H) ? ? ?? (3H / 4) ? ? 0. 4545\$? H 2? ? so the positive reactivity insertion is -0. 4545\$-(-0. 5\$)= 0. 04545\$?? (x) ? ?? (H) ? b) The rate of reactivity per cm can be found by differentiating the reactivity equation over the distance. ? 1 1 ? d ?? (x) d ? 1 ? x ?? ? ? ?? (H) ? ? Sin(2? x / H) ? ? ?? (H) ? ? Cos(2? x / H) ? dx dx ? ? H H ? ? H 2? ?? ? d ?? (x) ? 0. 005\$ / cm ? 0. cent / cm dx x ? 3H / 4 7. 31 There is a decrease in T so let's examine the effects of sign of temperature coefficients, If ? T ? (?) decrease in T ? decrease in k ? reduces P ? gives further dec. in k ? shut down(unstable) If ? T ? (?) decrease in T ? increase in k ? increase in P ? inc. in T and finally reactor returns to its original state! (stable) 7. 33 ? N FVF I ? p ? exp ? ? ? ? ? M ? sM VM ? I: Resonance Integral ? sM : Scattering Cross-Section of Moderator ? M : Constant 2a ? 1. 5 ? a ? 0. 75 (rod radius) dl I (300 K) ? 1 ? ? I (T) ? I (300 K)(1 ? ? 1 (T ? T0)) dT 2T I (T) ? ? ? sM ? M VM ln p N FVF T ? T0 ?

I (T) ? I (T0) ? ? k ln 0. 912 ? 0. 0921k where k ? ? sM ? M VM N FVF For slightly enriched uranium dioxide reactor take ? ? 10. 5 g / cm³ (See Chapter 6). ? 1 ? A? ? C? / a? where A? ? 61? 10? 4 and C? ? 2. 68 ? 10? 2 (Table 7. 4) ? ? 1 ? 0. 009503 T ? 665? C (? 938K) ? I (T) ? I (T0)(1 ? 13. 31* ? 1) ? 1. 1264I (T0) ? I (T) ? 0. 0921? 1. 1264 ? k ? 0. 1037k ? 1 ? ? k ? 665o C ? exp ? ? I (T) ? ? exp ? ? 0. 1037 ? ? 0. 9014 ? k ? ? k ? 7. 34 70 F ? 210C 550 F ? 287 0C d ? ?? ? T ? ? ? ?? ? (287 ? 21) ? ? 2 ? 10? 5 0C dT ? T where ? = 0. 0065 ? 1 ? ? 5. 32e ? 3 ? ? 0. 532% ? ? 0. 81\$ 7. 37 First you should solve problem 7. 6 to find the fraction of expelled water, 575F ? 301 0 C 585F ? 307 C 0 Vvessel ? 6 0 C increase in T ? D 2 ? ? 6. 5m3 ? Vwater ? v 0 ? 3. 25m3 4 ? v ? ? v ? T ? ? v ? 3. 25m3 ? 3e ? 3 ? 6 0 C ? 5. 85e ? 2m3 v0 ?? ? v ? 0. 018 v0 Then find f after expelling, k ? , 0 ? ?? pf 0 , critical state k ? , 1 ? ?? pf1 , original state ?? k ? , 1 ? 1 k ? , 1 ? k ? , 0 k ? , 1 ? ?? pf1 ? ?? pf 0 f ? 1? 0 ?? pf1 f1 ? a1F ? a 0 F f0 ? and we know ? a1F = 0. 95 ? a 0 F and finally, F M F M ? a1 ? ? a ? a 0 ? ? a f1 ? f0 1 0. 95? a 0 F ? ? a M 1? ? 1? () f1 0. 95 ? a 0 F ? ? a M f0 ? ? a F ? a F ? ? a M f? in here f 0 ? 0. 682 so ? a F ? a F ? 1 ? ?) ? a M ? a M 1 ? ? 1 ? a F f0 so f? 1 1 1 ? 0. 0982 ? (? 1) f0 ? 0. 956 f-f 0 ? 0. 287 f ?? 0. 287 Finally, ? T (f) ? ? 0 ? 0. 0478per 0 C ? T 6C Then ?? = LAMARSH SOLUTIONS CHAPTER-7 PART-4 7. 39 The reactivity equivalent of equilibrium xenon is to be; ? ? ? ? I ? ? X ? T where ? X ? 0. 770 ? 1013 / cm² ? sec and ? X ? 0. 00237 and ? I ? 0. 0639 ? p? ? X ? ? T ? ? 2. 42 and p ? ? ? 1 0 -0. 005 reactivity -0. 01 -0. 015 -0. 02 X: 4. 8 Y: -0. 02695 -0. 025 -0. 03 0 0. 5 1 1. 5 Note the convergence 2 2. 5 3 thermal flux x 1e14 3. 5 4 4. 5 5 7. 42 For Xenon using eq. 7. 94 X? ? (? I ? ? X)? f ? T ? X ? ? aX ? T here ? I ? 6. 39e ? 2 and ? X ? 2. 37e ? 3 (from table 7. 5) ? X ? 2. 09e ? 5 (from table 7. 6) You should make a

correction to the thermal absorption cross section as follows, ? 20 0. 5) 2
 200 ? aX (200? C) ? 0. 886 ? 1. 236 ? 2. 65e6 ? 1e ? 24 ? 0. 316 ? a, X ? ? g
 aXe (200 0C) ? ? a, X (20 0C) ? (? aX (200? C) ? 9. 17e ? 19cm 2 ? 9. 17e5b
 finally, X? ? 0. 06627 ? ? f ? 1e13 2. 09e ? 5 ? 9. 17e5b ? 1e13 For Samarium
 using eq. 7. 94 S? ? ? P ? f ? aX where ? P ? 0. 01071 ? 20 0. 5) 2 200 ? aX
 (200? C) ? 0. 886 ? 2. 093 ? 41e3 ? 1e ? 24 ? 0. 316 ? a, S ? S ? g a (200 0C)
 ? ? a, S (20 0C) ? (? aX (200? C) ? 2. 9e4b finally, S? ? 0. 01071 ? f 2. 39e4b

Note: When finding fission cross sections you should find the atom density of uranium 235 for this infinite thermal reactor. To do this , refer to example 6.

5 on page 294 taking buckling zero and find a relation between moderator number density and fuel density. 7. 43 Using eq. 7. 98 0. 06627 1e13 ? 2. 42
 1e13 ? 0. 773e13 where p=? = 1 0. 01071 ?? 2. 42 ? Xe ? ? ? Sm 7. 44 First
 of all, we must write the rate equations for each element; dN Sm ? ?? Sm N
 Sm ? ? a Sm N Sm? T ? ? Sm ? f ? T dt dN Eu ? ? Sm N Sm ? ? Eu N Eu ? ? a Eu
 N Eu? T dt dN Gd ? ? Eu N Eu ? ? a Gd N Gd? T dt) For equilibrium reactivity;
 N (t) ? N (t ? dt) ? Xi Xi and ignore ? a Sm N Sm? T & ? a Eu N Eu? T Inserted
 into all rate equations; N Sm ? Sm ? f ? T ? ? Sm dN X i (t) ? 0 dt ? Sm N Sm ?
 ? Eu N Eu ? a N Gd Gd ? Eu N Eu ? ? T Reactivity equation is found as
 below; ?? ?? where ? a Gd / ? f ?? p ? Sm ?? ?? p ? Sm ? 7 ? 10? 5 and ? ? 2.

42 and ? ? p ? 1 ? ? ? ? 2. 893 ? 10? 5 b) 157 Sm decays rapidly relative to
 157 Eu and half-life of the 157 Sm is too small so, dN Sm ? 0 ? ?? Sm N
 Sm ? ? Sm? f ? T ? ? Sm N Sm ? ? Sm? f ? T dt This equation is inserted into
 rate equation of 157 Eu and 157 Gd ; dN Eu ? ? Sm ? f ? T ? Eu N Eu dt dN Gd
 (t) ? ? Eu N Eu ? ? a Gd N Gd? T dt Gd At shutdown ? NOEu & NO are equal to
 equilibrium concentration for 157 Eu and 157Gd . ? No fission & no
 absorption is observed. From rate equation of Eu ? N

157 157 Gd Eu ? N Eu ? ? Eu t 0 (t) ? N e Gd (t) ? N Gd 0 ? Sm ? f ? T ?? Eu t ? e ? Eu ? Sm ? f ? T Eu ? (1 ? e?? t) ? Eu From equilibrium of Gd ? N 157 Gd 0 ? Sm ? f ?? a Gd ? Sm ? f ? Sm ? f ? T Eu ? N (t) ? ? (1 ? e?? t) ? a Gd ? Eu Gd Maximum reactivity is reached at time goes to infinity! Gd ? N max (t ? ?) ? ? Sm? f (?? ?? ? a Gd / ? f ?? p 1 ? a ? ? T) ? Eu Gd Sm where ? a ? ? f (1 ? ? T ? a Gd ? ? ? ? (1 ?) /? ? Eu Sm Gd where ? T ? a Gd) ? Eu ? Eu ? 1.

162 ? 10? 5 s ? ? ? ? 4. 386 ? 10? 5 ? ? 0. 675cents 7. 47 a) For constant power; $P = ER = fF(r, t)T(r, t)dV$ So as N decreases, flux should increase to keep power constant, $dN F(t) = aF \cdot T(t) (1) dt$

$$\frac{dN}{dt} = aF \cdot T(t)$$
$$\int_{N(0)}^{N(t)} dN = a \int_0^t F(t') T(t') dt'$$
$$N(t) - N(0) = a \int_0^t F(t') T(t') dt'$$
$$N(t) = N(0) + a \int_0^t F(t') T(t') dt'$$

b) $P = ER = fF(t)T(t)T(t) = P_{ER}fF(t)T(t)$

$$P = P_{ER}fF(t)T(t)$$
$$P = P_{ER}fF(N(0))T(0)t$$
$$P = P_{ER}fF(N(t))T(t)$$
$$P = P_{ER}fF(N(0))T(0)t$$